



U.S. Nuclear Waste Technical Review Board

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Nuclear Waste Assessment System for Technical Evaluation (NUWASTE)

Presented to:
Swedish National Council for Nuclear Waste

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Nuclear Waste – Burden or Benefit

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Topics

- NUWASTE Overview
 - Objective
 - Principles
 - Structure
- Differences Between US and Swedish Programs
- Swedish Evaluation
 - Scenarios
 - Disposal container savings
 - Natural uranium savings
- Typical refueling requirements
- Summary of results
- Scenario 4.1 processing projection
- Observation



NUWASTE Objectives

- Understand the impacts of potential fuel cycle initiatives on the generation and management of SNF, HLW and other radioactive waste streams.
- Create ability to vary system inputs to represent different initiatives that DOE may consider.
- Evaluate the impact on selected parameters, such as:
 - Number of surface dry storage casks required
 - Number of disposal waste packages required
 - Mass of natural uranium required
 - New waste streams
 - Proliferation risk
 - Facility utilization



NUWASTE Principles

- Focused on present light water reactor and reprocessing technology (single recycle).
- Covers the full life cycle of US nuclear power production and waste disposition.
- Based on material balances of assemblies and masses.
- Built on fundamental physics concepts and methods.
- Utilizes data from open literature and DOE documents.
- Deterministic methodology that enables the user to explore the sensitivity to various inputs.





- 12 facilities
- 10 waste streams

Tracks:

- 65 individual isotopes
- Fresh uranium BWR & PWR assemblies
- Recycled uranium BWR and PWR assemblies
- PWR and BWR MOX assemblies
- Mass & assemblies reprocessed
- Mass & assemblies disposed
- Waste stream masses

Tracks:

- 12 facilities
- 10 waste streams

tasks:

- 65 individual isotopes
- Fresh uranium BWR & PWR assemblies
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- PWR and BWR MOX assemblies
- Mass & assemblies reprocessed
- Mass & assemblies disposed
- Waste stream masses

Differences between the US and Swedish Programs

US assumptions based on Yucca Mountain Program, Swedish Assumptions based on SKB Technical Report TR-10-13

- Storage methodology
 - USA – Dry storage systems as necessary based on fuel pool capacity
 - Sweden – Transferred to CLAB after 2 years out-of-reactor

	Installed Nuclear Generation (GWe)	Inventory		Disposal Container Capacity			Disposal Criteria
		MT	Assemblies	BWR	PWR	HLW	
US	100	170,000	575,000	44	21	6	2.3 KW/m
Sweden	10	11,000	54,000	12	4	1	1.7 KW per container



Scenarios Evaluated

- Four datasets evaluated, each with 8 scenarios

Dataset	Scenario	Waste Stream	Processing Capacity
1	1.1 thru 1.8	No Life Extension	300 MT/year
2	2.1 thru 2.8	No Life Extension	500 MT/year
3	3.1 thru 3.8	10 Year Life Extension	300 MT/year
4	4.1 thru 4.8	10 Year Life Extension	500 MT/year

- Parameters evaluated
 - Disposal container savings
 - Natural uranium savings



Scenarios Evaluated

Scenarios for Dataset 1 and Dataset 3 – 300 MTU/year transportation capacity

Sequence No.	Disposal Start Year	Reprocessing	
		Start Year	Capacity (MT)
1	2023	2020	200
2		2025	
3		2020	150
4		2025	
5		2020	100
6		2025	
7		2020	50
8		2025	



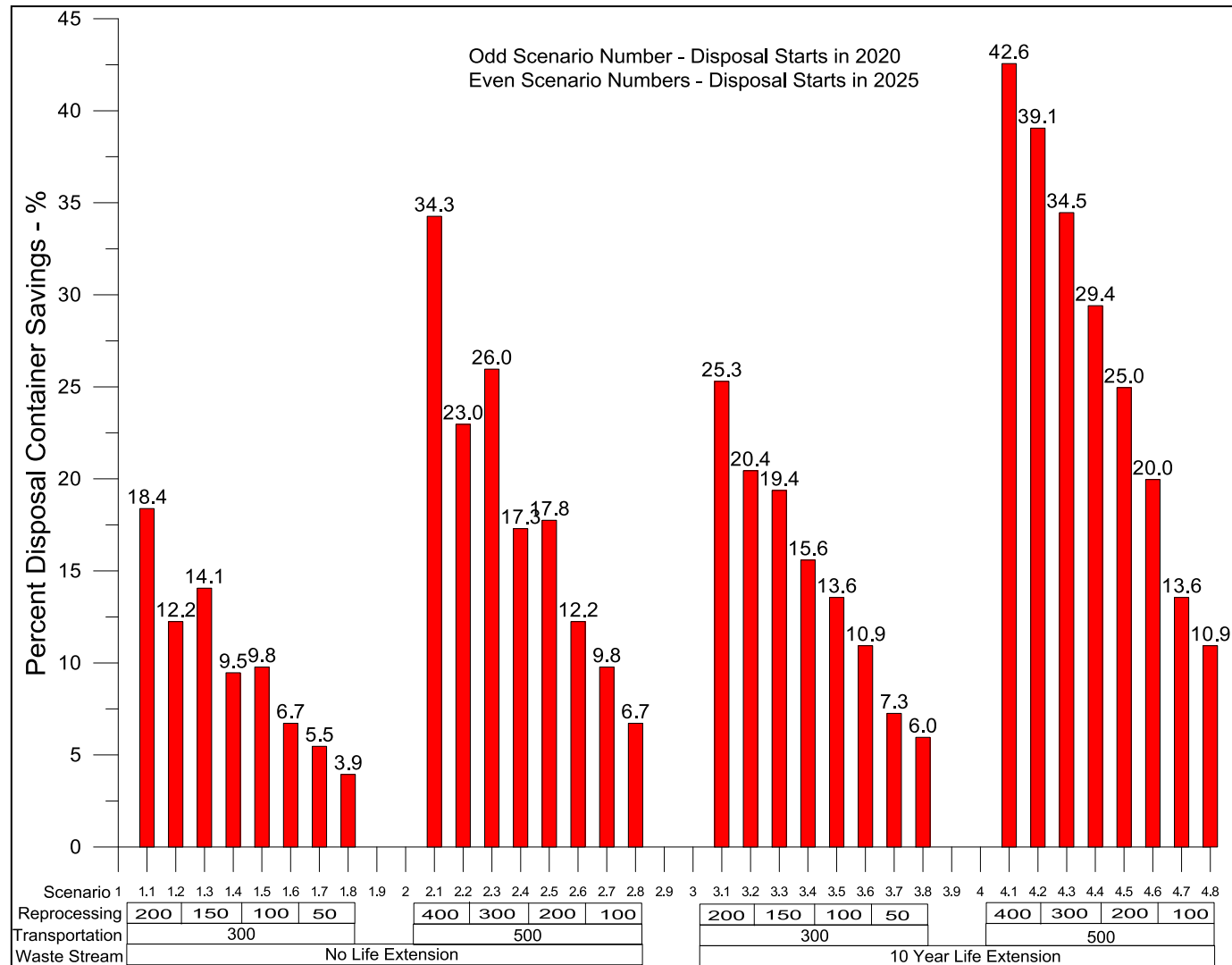
Scenarios Evaluated

Scenarios for Dataset 2 and Dataset 4 – 500 MTU/year transportation capacity

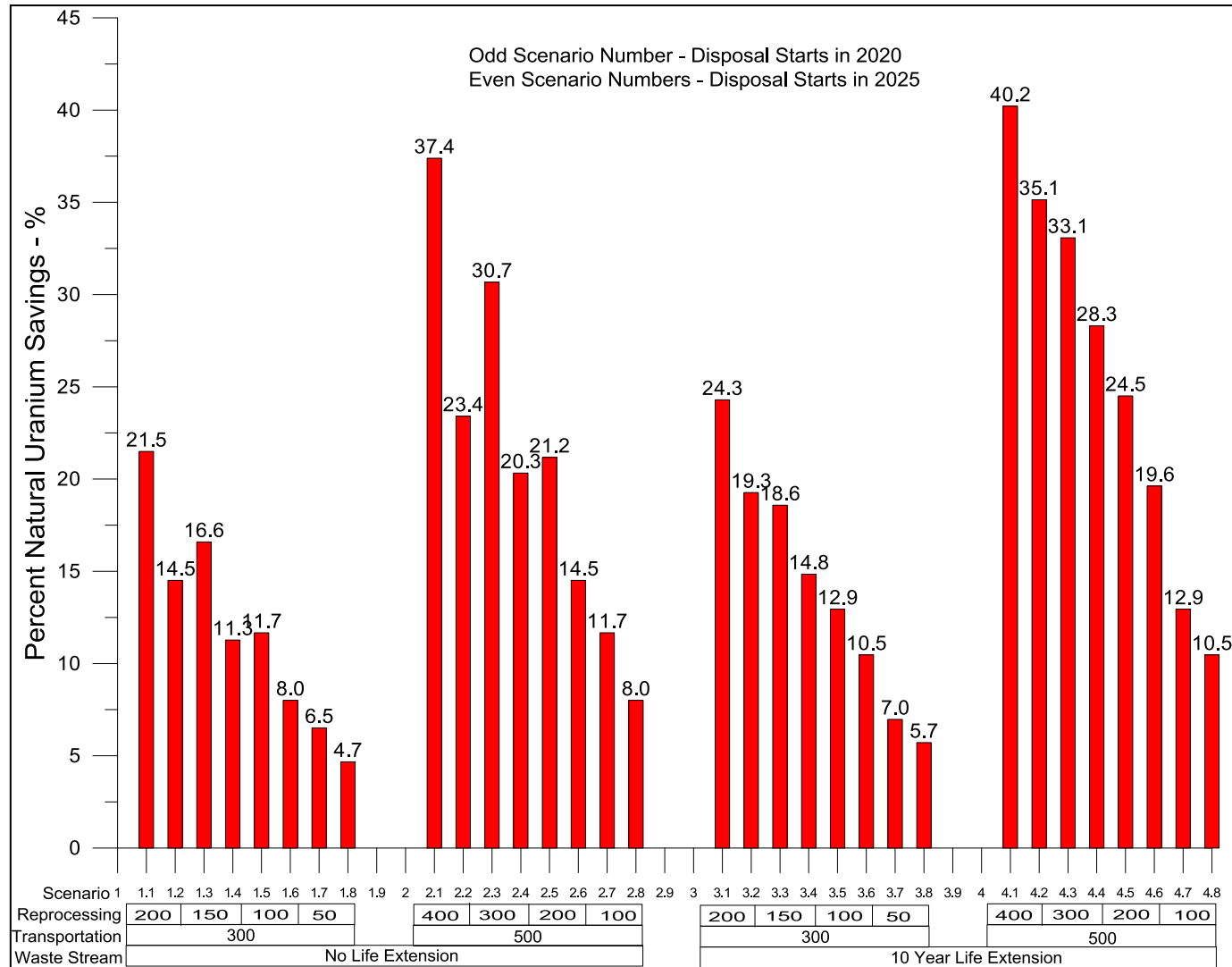
Sequence No.	Disposal Start Year	Reprocessing	
		Start Year	Capacity (MT)
1	2023	2020	400
2		2025	
3		2020	300
4		2025	
5		2020	200
6		2025	
7		2020	100
8		2025	



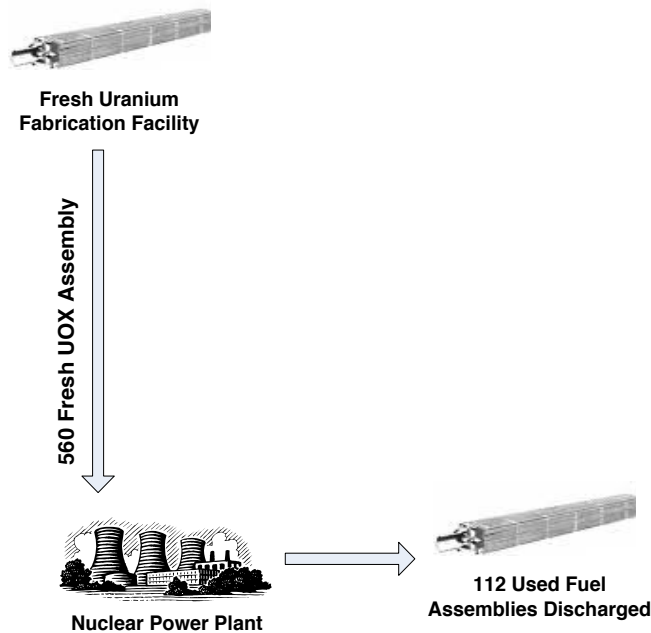
Disposal Container Savings



Natural Uranium Savings



Typical Refueling Requirement – Fresh Uranium Fuel



- BWR reactor core containing 560 fuel assemblies
- Average 20% assemblies discharges per year



Typical Yields from Reprocessing

Plutonium Yield

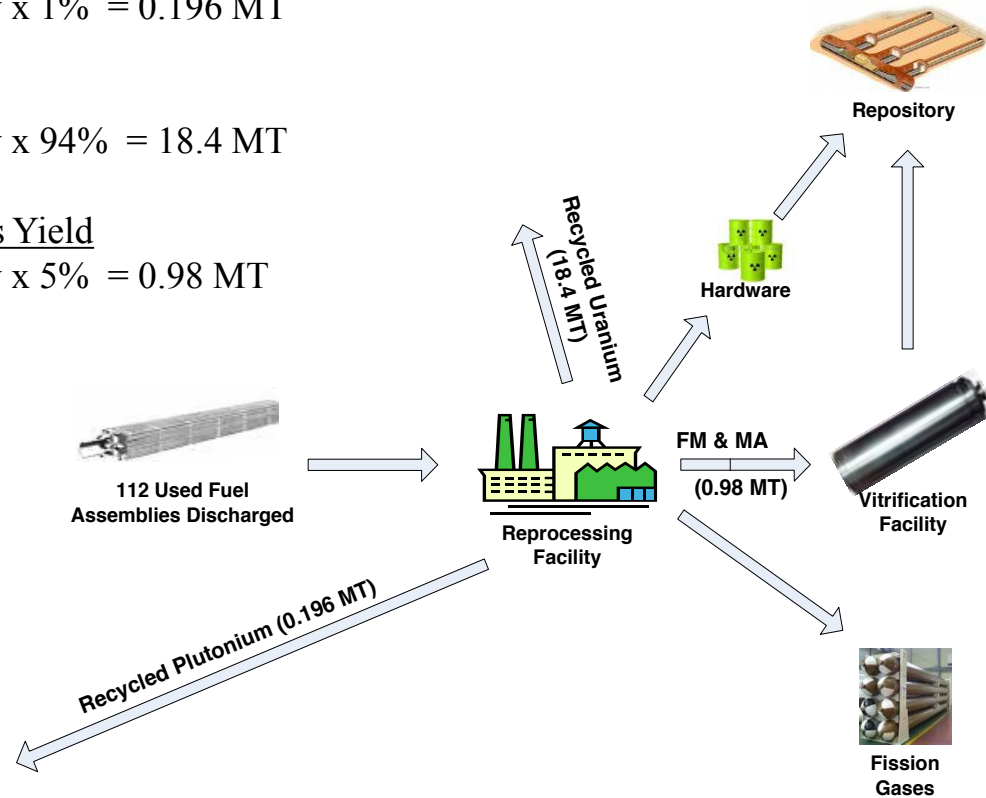
112 assemblies x 0.175 MT/assembly x 1% = 0.196 MT

Uranium Yield

112 assemblies x 0.175 MT/assembly x 94% = 18.4 MT

Fission Products and Minor Actinides Yield

112 assemblies x 0.175 MT/assembly x 5% = 0.98 MT



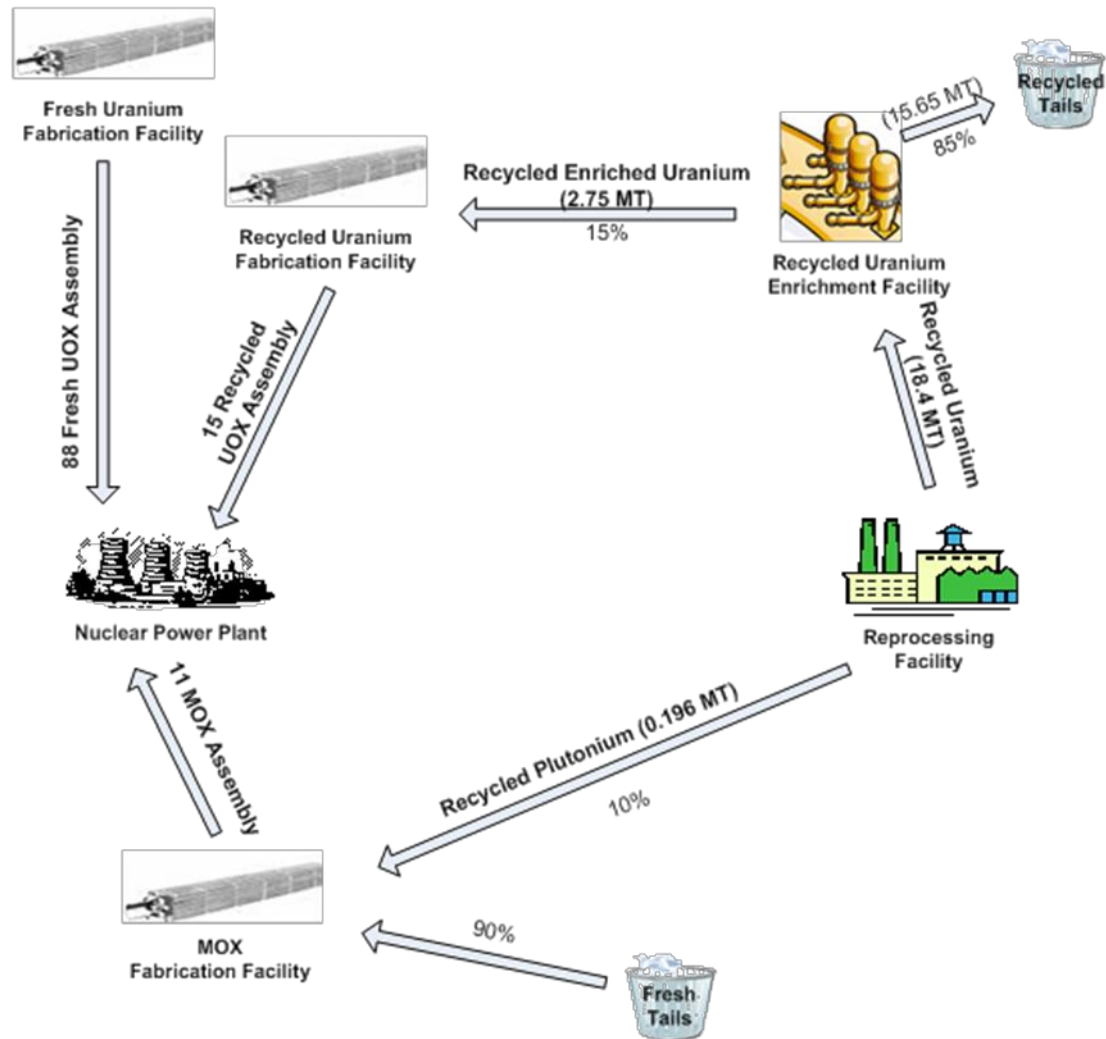
Fresh and Recycled Fuel Fabricated

112 assemblies needed to refuel reactor

- 15 recycled UOX
- 11 MOX
- 86 fresh UOX

Reduction in fresh uranium required for a single year

- $(15+11)/112 = 23\%$

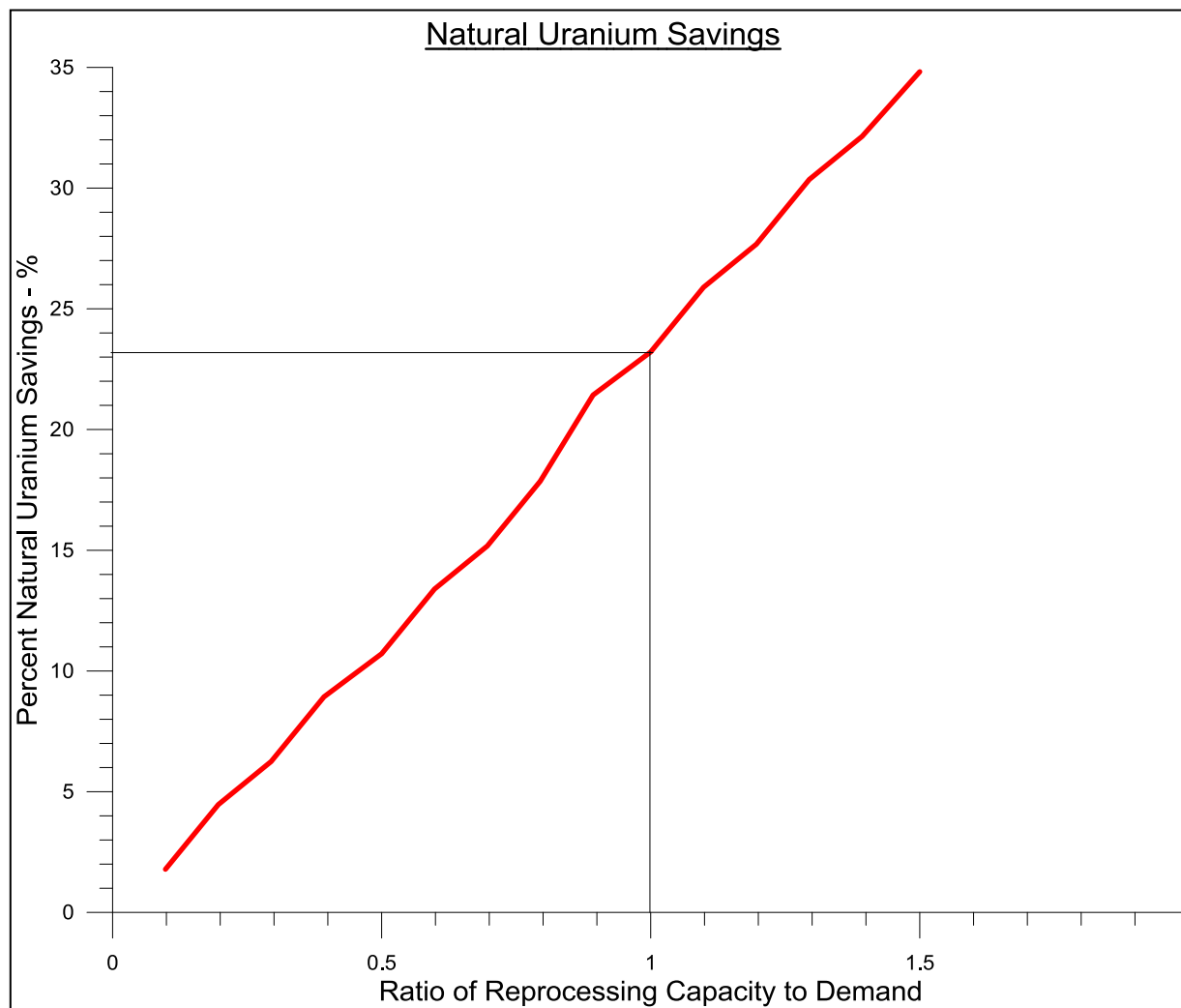


Natural Uranium Saving

A	B	C	D	E	F	G	H	I	J	K	L	M
Core Size	% Discharged Each Cycle	Number Discharged Assemblies (A x B)	% Discharged Assemblies Recycled	Number Recycled Assemblies (C x D)	MT per Assembly	MT Recycled Uranium (94%) (E x F x 0.94)	MT Enriched Recycled Uranium (15%) (G x 0.15)	Number Recycled UOX Assemblies (H/F)	MT Recycled Plutonium (1%) (E x F x 0.01)	Number Recycled MOX Assemblies (J/(F x 0.1))	Ratio of Reprocessing Capacity to Demand (E/C)	% Natural Uranium Savings (I + K)/C
560	20%	112	150%	168	0.175	27.64	4.15	23	0.29	16	1.50	34.8%
560	20%	112	140%	156	0.175	25.66	3.85	21	0.27	15	1.39	32.1%
560	20%	112	130%	145	0.175	23.85	3.58	20	0.25	14	1.29	30.4%
560	20%	112	120%	134	0.175	22.04	3.31	18	0.23	13	1.20	27.7%
560	20%	112	110%	123	0.175	20.23	3.04	17	0.22	12	1.10	25.9%
560	20%	112	100%	112	0.175	18.42	2.76	15	0.20	11	1.00	23.2%
560	20%	112	90%	100	0.175	16.45	2.47	14	0.18	10	0.89	21.4%
560	20%	112	80%	89	0.175	14.64	2.20	12	0.16	8	0.79	17.9%
560	20%	112	70%	78	0.175	12.83	1.92	10	0.14	7	0.70	15.2%
560	20%	112	60%	67	0.175	11.02	1.65	9	0.12	6	0.60	13.4%
560	20%	112	50%	56	0.175	9.21	1.38	7	0.10	5	0.50	10.7%
560	20%	112	40%	44	0.175	7.24	1.09	6	0.08	4	0.39	8.9%
560	20%	112	30%	33	0.175	5.43	0.81	4	0.06	3	0.29	6.3%
560	20%	112	20%	22	0.175	3.62	0.54	3	0.04	2	0.20	4.5%
560	20%	112	10%	11	0.175	1.81	0.27	1	0.02	1	0.10	1.8%



Natural Uranium Saving

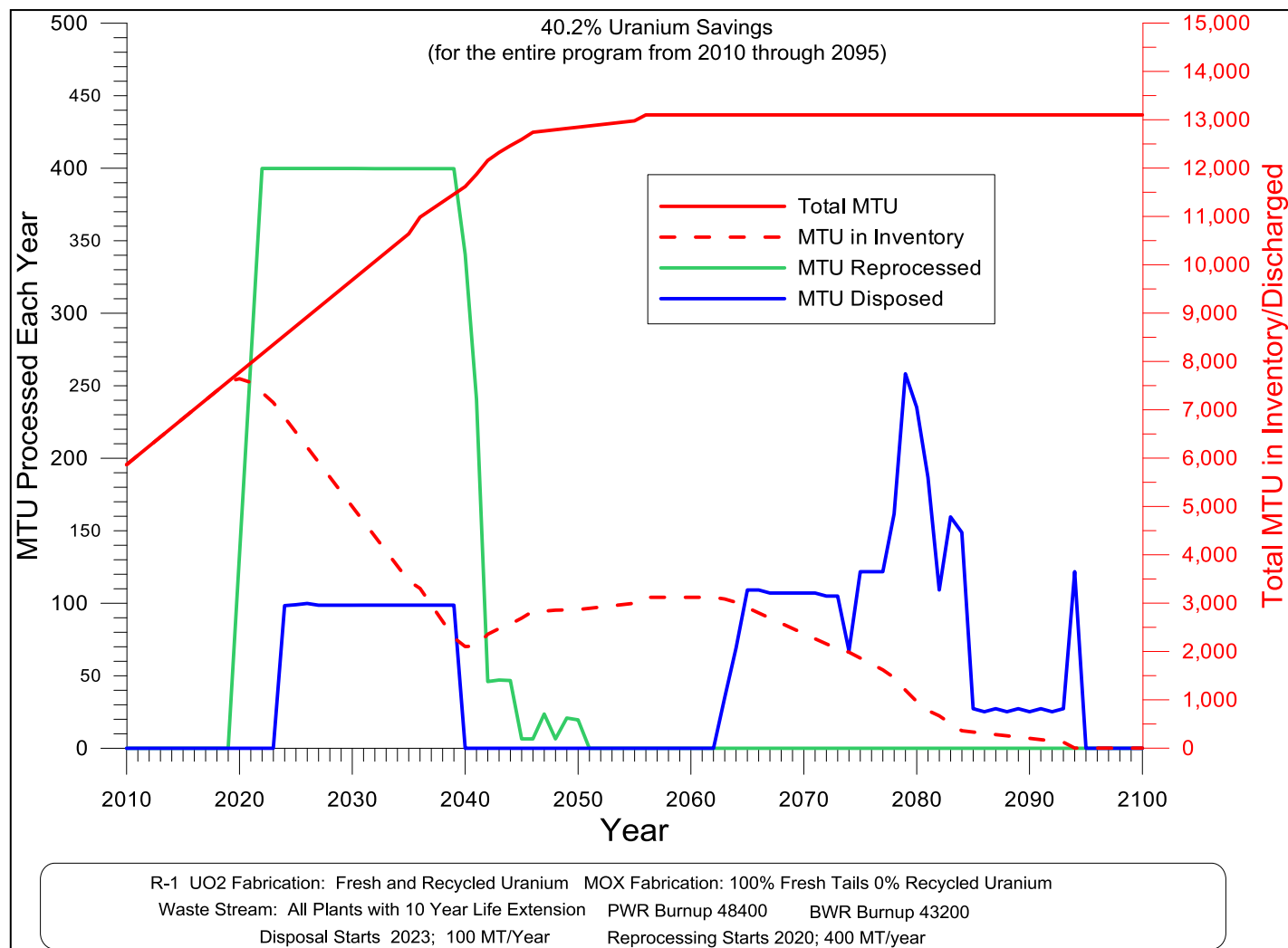


Summary of Disposal Container and Natural Uranium Savings

DataSet	Waste Stream	Disposal	Transportation	Reprocessing		Disposal Container Savings (%)	Natural Uranium Savings (%)
		Start	Capacity (MT/year)	Start	Capacity (MT/year)		
1.1	No Life Extension	2023	300	2020	200	18.39	21.50
1.2		2023		2025	200	12.25	14.51
1.3		2023		2020	150	14.06	16.59
1.4		2023		2025	150	9.47	11.28
1.5		2023		2020	100	9.77	11.67
1.6		2023		2025	100	6.72	8.01
1.7		2023		2020	50	5.47	6.51
1.8		2023		2025	50	3.94	4.67
2.1		2023	500	2020	400	34.27	37.39
2.2		2023		2025	400	22.97	23.41
2.3		2023		2020	300	25.96	30.67
2.4		2023		2025	300	17.30	20.33
2.5		2023		2020	200	17.75	21.17
2.6		2023		2025	200	12.25	14.51
2.7		2023		2020	100	9.77	11.67
2.8		2023		2025	100	6.72	8.01
3.1	10 Year Life Extension	2023	300	2020	200	25.30	24.29
3.2		2023		2025	200	20.44	19.27
3.3		2023		2020	150	19.37	18.58
3.4		2023		2025	150	15.59	14.85
3.5		2023		2020	100	13.56	12.94
3.6		2023		2025	100	10.94	10.47
3.7		2023		2020	50	7.26	6.97
3.8		2023		2025	50	5.96	5.72
4.1		2023	500	2020	400	42.56	40.21
4.2		2023		2025	400	39.06	35.14
4.3		2023		2020	300	34.47	33.08
4.4		2023		2025	300	29.41	28.31
4.5		2023		2020	200	24.96	24.51
4.6		2023		2025	200	19.96	19.62
4.7		2023		2020	100	13.56	12.94
4.8		2023		2025	100	10.94	10.47



Scenario 4.1 Processing Projection



Observations

- There is no optimum scenario:
 - Optimization depends on which parameters are important.
 - Conflicting parameters exist (maximizing uranium savings would result in increased cost).
- Uranium savings depends on:
 - Reactor operating Lifetimes.
 - Reprocessing capacity.
 - Reprocessing operating period.
 - Number of assemblies disposed prior to completing reprocessing (assemblies available for reprocessing).
 - New assembly demand.
- Facility capacity utilization depends on timing and capacity of facilities.

